

【海外研究成果】 炉内溶融物の冷却に関する文献リスト(INIS)

※ 文献複写をお申し込みの際は「登録番号」、「標題」、「著者情報」(第1著者名のみ可)を明記願います。

No.	登録番号	標題	著者情報	情報源	出版地	発行年
1	43009359	Fukushima Dai-ichi power plant accident - August 5, 2011 status	Institut de Radioprotection et de Surete Nucleaire, IRSN, 92 - Fontenay-aux-Roses (France)	2011; 3 p.; INIS-FR--12-0039; Also available from the INIS Liaison Officer for France, see the 'INIS contacts' section of the INIS-NKM website for current contact and E-mail addresses: http://www.iaea.org//inis/Contacts/index.htm ;	France	2011
2	43038487	Criticality Analysis of Corium and Cooling Water Mixture	Kim, Jong Kyung; Shin, Chang Ho; Kim, Hong Chul; Kim, Song Hyun; Park, Jung Min; KAERI, Daejeon (Korea, Republic of); Korea Atomic Energy Research Institute, Daejeon (Korea, Republic of)	May 2011; 99 p.; KAERI/CM--1416/2010; . Also available from KAERI; 2 refs, 23 figs, 24 tabs;	Korea, Republic of	2011
3	43009355	Situation of Fukushima Dai-ichi power plant in Japan - May 25, 2011 status	Institut de Radioprotection et de Surete Nucleaire, IRSN, 92 - Fontenay-aux-Roses (France)	2011; 3 p.; INIS-FR--12-0035; Also available from the INIS Liaison Officer for France, see the 'INIS contacts' section of the INIS-NKM website for current contact and E-mail addresses: http://www.iaea.org//inis/Contacts/index.htm ;	France	2011
4	43045561	Preliminary Analysis of Corium Behavior in the Lower Plenum by using ASTEC Computer Code	Park, Rae Joon; Kang, Kyoung Ho; Ahn, Kwang Il; Hong, Seong Wan; Korea Atomic Energy Research Institute, Daejeon (Korea, Republic of)	Korean Nuclear Society, Daejeon (Korea, Republic of); KNS; Daejeon (Korea, Republic of); Proceedings of the KNS autumn meeting; [1 CD-ROM]; Oct 2011; [2 p.]; 2011 autumn meeting of the KNS; Kyoungju (Korea, Republic of); 26-28 Oct 2011; Available from KNS, Daejeon (KR); 2 refs, 6 figs;	Korea, Republic of	2011
5	42095280	Analysis of molten core concrete interaction during severe accidents in PHWR	Jagannad, VBL; Reddy, VV; Hajela, Sameer; Kansal, M; Rammohan, HP; Malhotra, PK; Ghadge, SG; jagan@npcilcoin; Reactor Safety and Analysis, Nuclear Power Corporation of India Limited, Mumbai (India)	Fourth national conference on nuclear reactor technology: emerging trends in nuclear safety; Mar 2011; [9 p.]; NRT-4: 4. national conference on nuclear reactor technology; Mumbai (India); 4-6 Mar 2011; 9 refs., 17 figs., 1 tab.;	India	2011

No.	登録番号	標題	著者情報	情報源	出版地	発行年
6	43009360	Fukushima Dai-ichi power plant accident - August 25, 2011 status	Institut de Radioprotection et de Surete Nucleaire, IRSN, 92 - Fontenay-aux-Roses (France)	2011; 4 p.; INIS-FR--12-0040; Also available from the INIS Liaison Officer for France, see the 'INIS contacts' section of the INIS-NKM website for current contact and E-mail addresses: http://www.iaea.org//inis/Contacts/index.htm ;	France	2011
7	42099204	Analysis of Air-Water Two Phase Flow for K-HERMES-HALF Experiment using RELAP5/MOD3	Park, Rae Joon; Ha, Kwang Soon; Kim, Sang Baik; Hong, Seong Wan; Korea Atomic Energy Research Institute, Daejeon (Korea, Republic of); Heo, Sun; KHNP Nuclear Engineering and Technology Institute, Daejeon (Korea, Republic of)	Korean Nuclear Society, Daejeon (Korea, Republic of); KNS; Daejeon (Korea, Republic of); Proceedings of the KNS spring meeting; [1 CD-ROM]; May 2011; [2 p.]; 2011 spring meeting of the KNS; Taebaek (Korea, Republic of); 26-27 May 2011; Available from KNS, Daejeon (KR); 2 refs, 5 figs;	Korea, Republic of	2011
8	43009356	Situation of Fukushima Dai-ichi power plant in Japan - June 1, 2011 status	Institut de Radioprotection et de Surete Nucleaire, IRSN, 92 - Fontenay-aux-Roses (France)	2011; 3 p.; INIS-FR--12-0036; Also available from the INIS Liaison Officer for France, see the 'INIS contacts' section of the INIS-NKM website for current contact and E-mail addresses: http://www.iaea.org//inis/Contacts/index.htm ;	France	2011
9	43020312	What happened in the reactor fuels of Fukushima Daiichi nuclear power plants Nuclear fuel behavior when core is exposed	Fujishiro, Toshio; Research Organization for Information Science and Technology, Tokai, Ibaraki (Japan)	Nippon Genshiryoku Gakkai-Shi; Aug 2011; p. 550-554; v. 53(8); 5 refs., 4 figs., 1 tab.;	Japan	2011
10	43009357	Fukushima Dai-ichi power plant accident - June 10, 2011 status	Institut de Radioprotection et de Surete Nucleaire, IRSN, 92 - Fontenay-aux-Roses (France)	2011; 3 p.; INIS-FR--12-0037; Also available from the INIS Liaison Officer for France, see the 'INIS contacts' section of the INIS-NKM website for current contact and E-mail addresses: http://www.iaea.org//inis/Contacts/index.htm ;	France	2011
11	43009358	Fukushima Dai-ichi power plant accident - July 6, 2011 status	Institut de Radioprotection et de Surete Nucleaire, IRSN, 92 - Fontenay-aux-Roses (France)	2011; 3 p.; INIS-FR--12-0038; Also available from the INIS Liaison Officer for France, see the 'INIS contacts' section of the INIS-NKM website for current contact and E-mail addresses: http://www.iaea.org//inis/Contacts/index.htm ;	France	2011

No.	登録番号	標題	著者情報	情報源	出版地	発行年
12	43012266	Research and development with regard to severe accidents in pressurised water reactors: Summary and outlook	Institut de Radioprotection et de Surete Nucleaire, IRSN, 92 - Fontenay-aux-Roses (France)	2011; 218 p.; IRSN--2007-83; CEA--2007-351; 290 refs.; Also available from the INIS Liaison Officer for France, see the 'INIS contacts' section of the INIS-NKM website for current contact and E-mail addresses: http://www.iaea.org//inis/Contacts/index.htm ;	France	2011
13	42085222	CHF Experiments under Realistic Severe Accident Condition for IVR-ERVC Strategy	Kim, Tae Il; Park, Hae Min; Chang, Soon Heung; Korea Advanced Institute of Science and Technology, Daejeon (Korea, Republic of); Heo, Sun; Korea Hydro and Nuclear Power Co, Daejeon (Korea, Republic of)	Korean Nuclear Society, Daejeon (Korea, Republic of); KNS; Daejeon (Korea, Republic of); Proceedings of the KNS autumn meeting; [1 CD-ROM]; Oct 2010; [2 p.]; 2010 autumn meeting of the KNS; Jeju (Korea, Republic of); 21-22 Oct 2010; Available from KNS, Daejeon (KR); 1 ref, 4 figs, 1 tab;	Korea, Republic of	2010
14	41086791	Corium Behaviour and the Lower Head Thermal Response after a Core Meltdown	Sadek, S; Grgic, D; Debrecin, N; University of Zegrab, Faculty of Electrical Engineering and Computing, Zagreb (Croatia)	Croatian Nuclear Society (Croatia); Proceedings and Book of Abstracts of 8th International Conference: Nuclear Option in Countries with Small and Medium Electricity Grids; 122 p.; 2010; p. 76; 8th International Conference: Nuclear Option in Countries with Small and Medium Electricity Grids; Dubrovnik (Croatia); 16-20 May 2010; INIS-HR--10004;	Croatia	2010
15	42031397	On spreading of melt jets in the pool of volatile coolant	Moghaddam Vahid Hasani; Kazachkov, IV; National Technical University of Ukraine 'Kyiv Polytechnical Institut', Kyiv (Ukraine)	Yaderna ta Radyatsyijna Bezpeka; Mar-May 2010; p. 19-26; (no.2);	Ukraine	2010
16	41073160	In-vessel coolability and steam explosion in Nordic BWRs	Ma, W; Hansson, R; Li, L; Kudinov, P; Cadinu, F; Tran, C-T; Royal Institute of Technology (KTH), Stockholm (Sweden); Nordisk Kernesikkerhedsforskning, Roskilde (Denmark)	May 2010; 31 p.; NKS--219; ISBN 978-87-7893-289-1; . Also available at http://130.226.56.153/rispubl/NKS/NKS-219.pdf ; NKS-R-INCOSE; 6 tabs., 21 ill., 28 refs.;	Denmark	2010

No.	登録番号	標題	著者情報	情報源	出版地	発行年
17	42013403	In-vessel melt pool coolibility test-Description and results of LIVE experiments	Gaus-Liu, X; XiaoyangGaus-Liu@iketfzkde; Karlsruhe Institute of Technology, Hermann-von-Helmholtz-Platz-1, 76344 Eggenstein-Leopoldshafen (Germany); Miassoedov, A; AlexeiMiassoedov@iketfzkde; Karlsruhe Institute of Technology, Hermann-von-Helmholtz-Platz-1, 76344 Eggenstein-Leopoldshafen (Germany); Cron, T; ThomasCron@iketfzkde; Karlsruhe Institute of Technology, Hermann-von-Helmholtz-Platz-1, 76344 Eggenstein-Leopoldshafen (Germany); Wenz, T; ThomasWenz@iketfzkde; Karlsruhe Institute of Technology, Hermann-von-Helmholtz-Platz-1, 76344 Eggenstein-Leopoldshafen (Germany)	Nuclear Engineering and Design; Nov 2010; p. 3898-3903; v. 240(11); 10.1016/j.nucengdes.2010.09.001; S0029-5493(10)00537-6; Available from http://dx.doi.org/10.1016/j.nucengdes.2010.09.001 ; Copyright (c) 2010 Elsevier Science B.V., Amsterdam, The Netherlands, All rights reserved.; Country of input: International Atomic Energy Agency (IAEA);	Netherlands	2010
18	42098327	COCOSYS New modelling of safety relevant phenomena and components	Klein-Hessling, W; Nowack, H; Spengler, C; Weber, G; Hoehne, M; Sonnenkalb, M; Gesellschaft fuer Anlagen- und Reaktorsicherheit mbH (GRS), Koeln (Germany)	Gesellschaft fuer Anlagen- und Reaktorsicherheit mbH (GRS), Koeln (Germany); Institut de Radioprotection et de Surete Nucleaire (IRSN), Fontenay-aux-Roses (France); Bel V, Bruxelles (Belgium); EUROSAFE Forum 2010. Towards convergence of technical nuclear safety practices in Europe; 1086 p.; 2010; 16 p.; EUROSAFE Forum 2010; Koeln (Germany); 8-9 Nov 2010; Available from TIB Hannover;	Germany	2010
19	42098328	Review of 10 years of molten corium concrete interaction R and D Potential applications for containment integrity	Bonnet, JM; Institut de Radioprotection et de Surete Nucleaire (IRSN), Fontenay-aux-Roses (France); Spengler, C; Gesellschaft fuer Anlagen- und Reaktorsicherheit mbH (GRS), Koeln (Germany); Sevon, T; VTT Technical Research Centre of Finland, Espoo (Finland); Rydl, A; UJV, Nuclear Research Institute Rez plc, Rez (Czech Republic)	Gesellschaft fuer Anlagen- und Reaktorsicherheit mbH (GRS), Koeln (Germany); Institut de Radioprotection et de Surete Nucleaire (IRSN), Fontenay-aux-Roses (France); Bel V, Bruxelles (Belgium); EUROSAFE Forum 2010. Towards convergence of technical nuclear safety practices in Europe; 1086 p.; 2010; 26 p.; EUROSAFE Forum 2010; Koeln (Germany); 8-9 Nov 2010; Available from TIB Hannover;	Germany	2010
20	42012336	Criticality Analysis of Corium and Cooling Water Mixture	Kim, Jong Kyung; Shin, Chang Ho; Kim, Hong Chul; Kim, Song Hyun; Park, Jung Min; Hanyang University, Seoul (Korea, Republic of); Korea Atomic Energy Research Institute, Daejeon (Korea, Republic of)	May 2010; 66 p.; KAERI/CM--1245/2009; . Also available from KAERI; 12 refs, 20 figs, 33 tabs;	Korea, Republic of	2010
21	43027291	Development of next-generation light water reactor	Ishibashi, Fumihiko; Yasuoka, Makoto; Toshiba Corp, Power Systems Co, Tokyo (Japan)	Toshiba Rebyu; Dec 2010; p. 26-28; v. 65(12); 2 refs., 1 fig., 2 tabs.;	Japan	2010

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22	41079699	The COMET-L3 experiment on long-term melt-concrete interaction and cooling by surface flooding	Miassoedov, Alexei; alexemiassoedov@iketfzkde; Forschungszentrum Karlsruhe GmbH, Postfach 3640, 76021 Karlsruhe (Germany); Alsmeyer, Hans; Cron, Thomas; Foit, Jerzy; Forschungszentrum Karlsruhe GmbH, Postfach 3640, 76021 Karlsruhe (Germany)	Nuclear Engineering and Design; Feb 2010; p. 258-265; NURETH-12: 12. international topical meeting on nuclear reactor thermal hydraulics; Pittsburgh, PA (United States); 30 Sep - 4 Oct 2007; v. 240(2); 10.1016/j.nucengdes.2008.12.005; S0029-5493(08)00625-0; Available from http://dx.doi.org/10.1016/j.nucengdes.2008.12.005 ; Copyright (c) 2008 Elsevier Science B.V., Amsterdam, The Netherlands, All rights reserved.; Country of input: International Atomic Energy Agency (IAEA);	Netherlands	2010
23	42085151	Corium Retention and Cooling by an Ex-vessel Core-catcher for Nuclear Power Plants	Lee, Jong Ho; Kim, Ji Hwan; Korea Hydro and Nuclear Power, Daejeon (Korea, Republic of)	Korean Nuclear Society, Daejeon (Korea, Republic of); KNS; Daejeon (Korea, Republic of); Proceedings of the KNS autumn meeting; [1 CD-ROM]; Oct 2010; [2 p.]; 2010 autumn meeting of the KNS; Jeju (Korea, Republic of); 21-22 Oct 2010; Available from KNS, Daejeon (KR); 8 refs, 2 figs;	Korea, Republic of	2010
24	42085153	Critical Heat Flux Experiments for In-Vessel Retention External Reactor Vessel Cooling Strategy using 2-D Slice Test Section	Park, Hae Min; Kim, Tae Il; Jeong, Yong Hoon; Korea Advanced Institute of Science and Technology, Daejeon (Korea, Republic of); Heo, Sun; Korea Hydro and Nuclear Power Co, Daejeon (Korea, Republic of)	Korean Nuclear Society, Daejeon (Korea, Republic of); KNS; Daejeon (Korea, Republic of); Proceedings of the KNS autumn meeting; [1 CD-ROM]; Oct 2010; [2 p.]; 2010 autumn meeting of the KNS; Jeju (Korea, Republic of); 21-22 Oct 2010; Available from KNS, Daejeon (KR); 4 refs, 4 figs, 1 tab;	Korea, Republic of	2010
25	43018423	Overview on the experiments concerning passive safety systems of the AP1000 trademark	Freis, Daniel; Knoche, Dietrich; Tietsch, Wolfgang; Westinghouse Electric Germany GmbH, Mannheim (Germany)	Annual meeting on nuclear technology 2010. Documentation; May 2010; 6 p.; Annual meeting on nuclear technology 2010; Berlin (Germany); 4-6 May 2010;	Germany	2010
26	43036753	Ex-vessel corium coolability sensitivity study with the CORQUENCH code	Robb, Kevin; Corradini, Michael; University of Wisconsin-Madison, Madison, Wisconsin (United States)	Atomic Energy Society of Japan, Tokyo (Japan); NURETH-13: Proceedings of the 13th international topical meeting on nuclear reactor thermal hydraulics; [4617 p.]; 2009; [15 p.]; NURETH-13: 13. international topical meeting on nuclear reactor thermal hydraulics; Kanazawa, Ishikawa (Japan); 27 Sep - 2 Oct 2009; Available from Atomic Energy Society of Japan, 2-3-7, Shimbashi, Minato, Tokyo, 105-0004 JAPAN; Available as CD-ROM Data in PDF format, Folder Name: FullPaper, Paper ID: N13P1290.pdf;	Japan	2009

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27	41010312	ASTEC application to in-vessel corium retention	Tarabelli, D; Commissariat a l'Energie Atomique, Cadarache (France); Ratel, G; Pelisson, R; Commissariat a l'Energie Atomique, Grenoble (France); Guillard, G; Institut de Radioprotection et de Surete Nucleaire (IRSN), DPAM, Cadarache, Saint-Paul-lez-Durance, 13115 BP3 Cedex (France); Barnak, M; Inzinierska Vypoctova Spolocnost, Hviezdoslavova 12, 91701 Trnava (Slovakia); Matejovic, P; ivstt@nextask; Inzinierska Vypoctova Spolocnost, Hviezdoslavova 12, 91701 Trnava (Slovakia)	Nuclear Engineering and Design; Jul 2009; p. 1345-1353; v. 239(7); 10.1016/j.nucengdes.2009.02.021; S0029-5493(09)00131-9; Available from http://dx.doi.org/10.1016/j.nucengdes.2009.02.021 ; Copyright (c) 2009 Elsevier Science B.V., Amsterdam, The Netherlands, All rights reserved.; Country of input: International Atomic Energy Agency (IAEA);	Netherlands	2009
28	41027314	Evaluation Methodology of External Corium Cooling System	Ha, Kwang Soon; Kim, Hwan Yeol; Kim, Jong Tae; Park, Jong Hwa; Korea Atomic Energy Research Institute, Daejeon (Korea, Republic of)	Korean Nuclear Society, Daejeon (Korea, Republic of); KNS; Daejeon (Korea, Republic of); Proceedings of the KNS autumn meeting; [1 CD-ROM]; Oct 2009; [2 p.]; 2009 autumn meeting of the KNS; Kyungju (Korea, Republic of); 29-30 Oct 2009; Available from KNS, Daejeon (KR); 1 ref, 2 figs, 1 tab;	Korea, Republic of	2009
29	41061633	On the modeling of bending perturbations of melt jets under reactor vessel water pool during severe accident at NPP	Moghaddam, VH; Kazachkov, IA; National Technical University of Ukraine, 'KPI', Kyiv (Ukraine)	Yaderna Fyzika ta Energetika; Jul-Sep 2009; p. 293-298; v. 10(no.3);	Ukraine	2009
30	40101629	Overview of IRSN R and D activities on severe accidents	Chaumont, B; Raimond, E; bernardchaumont@irsnfr; Institut de Radioprotection et de Surete Nucleaire (IRSN), BP 17, Fontenay-aux-Roses Cedex (France); Dorsselaere, JP Van; Simondi-Teisseire, B; IRSN/DPAM, BP 3, Saint-Paul-lez-Durance, 13115 Cedex (France); Institut de Radioprotection et de Surete Nucleaire (IRSN), BP 17, Fontenay-aux-Roses Cedex (France)	Constantin, Marin; Turcu, Ilie; Institute for Nuclear Research-Pitesti, PO Box 78, 1 Campului Str., RO-115400 Mioveni, Arges (Romania); Institute for Nuclear Research-Pitesti, PO Box 78, 1 Campului Str., RO-115400 Mioveni, Arges (Romania); University of Pitesti, Bd. Republicii, 71, Pitesti (Romania); National Authority for Scientific Research, Bucharest (Romania); Institute for Nuclear Research - Pitesti; Pitesti (Romania); Proceedings of NUCLEAR 2009 international conference on sustainable development through nuclear research and education; 707 p.; 2009; p. 10-19; NUCLEAR 2009: 2. international conference on sustainable development through nuclear research and education; Pitesti (Romania); 27-29 May 2009; Available from author(s) or Institute for Nuclear Research-Pitesti, 1 Campului Str., RO-115400 Mioveni, Arges (RO); Available from Institute for Nuclear Research-Pitesti, 1 Campului Str., RO-115400 Mioveni, Arges (RO); University of Pitesti, Bd. Republicii, 71, Pitesti (RO); 11 refs., 6 figs. Talk given to the first plenary session;	Romania	2009

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31	41010334	A scoping study of debris bed formation in the DEFOR test facility	Karbojian, A; aram@safetysickthse; Division of Nuclear Power Safety, Royal Institute of Technology (KTH), 106 91 Stockholm (Sweden); Ma, WM; Kudinov, P; Dinh, TN; Division of Nuclear Power Safety, Royal Institute of Technology (KTH), 106 91 Stockholm (Sweden)	Nuclear Engineering and Design; Sep 2009; p. 1653-1659; ICONE 15: 15. international conference on nuclear engineering; Nagoya (Japan); 22-26 Apr 2007; v. 239(9); 10.1016/j.nucengdes.2009.03.002; S0029-5493(09)00141-1; Available from http://dx.doi.org/10.1016/j.nucengdes.2009.03.002 ; Copyright (c) 2009 Elsevier Science B.V., Amsterdam, The Netherlands, All rights reserved.; Country of input: International Atomic Energy Agency (IAEA);	Netherlands	2009
32	41010450	The peculiarities of corium melt spreading and fragmentation in the coolant pool under reactor vessel during severe accidents at NPP	Mogaddam, VKh; Kazachkov, IV; National Technical University of Ukraine 'Kyiv Polytechnical Institute', Kyiv (Ukraine)	Yaderna ta Radyatsyjna Bezpeka; Oct-Dec 2009; p. 61-68; v. 12(no.4);	Ukraine	2009
33	40082012	Evaluation of corium cooling in external core catcher system	Ha, Kang Soon; Kim, Hwan Yeol; Kim, Jong Tae; Park, Jong Hwa; KAERI, Daejeon (Korea, Republic of)	Korea Atomic Energy Research Institute, Daejeon (Korea, Republic of); Nuclear Power Institute of China (China); KAERI; Daejeon (Korea, Republic of); Proceedings for the Fourth Korea-China Workshop on Nuclear Reactor Thermal-Hydraulics (WORTH-4); 534 p.; 2009; p. 333-340; 4. Korea-China Workshop on Nuclear Reactor Thermal-Hydraulics; Jeju (Korea, Republic of); 18-20 May 2009; Available from KAERI, Daejeon (KR); 11 refs, 7 figs, 4 tabs;	Korea, Republic of	2009
34	43036650	Experimental results on the coolability of a debris bed with multidimensional cooling effect	Rashid, Muhammad; Kulenovic, Rudi; Laurien, Eckart; Institute for Nuclear Technology and Energy Systems (IKE), University of Stuttgart, Stuttgart (Germany); Nayak, AK; Reactor Engineering Division, Bhabha Atomic Research Centre, Mumbai (India)	Atomic Energy Society of Japan, Tokyo (Japan); NURETH-13: Proceedings of the 13th international topical meeting on nuclear reactor thermal hydraulics; [4617 p.]; 2009; [11 p.]; NURETH-13: 13. international topical meeting on nuclear reactor thermal hydraulics; Kanazawa, Ishikawa (Japan); 27 Sep - 2 Oct 2009; Available from Atomic Energy Society of Japan, 2-3-7, Shimbashi, Minato, Tokyo, 105-0004 JAPAN; Available as CD-ROM Data in PDF format, Folder Name: FullPaper, Paper ID: N13P1148.pdf;	Japan	2009
35	39065159	Ex-Vessel corium coolability and steam explosion energetics in nordic light water reactors	Dinh, TN; Ma, WM; Karbojian, A; Kudinov, P; Tran, CT; Hansson, CR; Royal Institute of Technology (KTH), (Sweden); Nordisk Kernesikkerhedsforskning, Roskilde (Denmark)	Mar 2008; 72 p.; NKS--160; KTH-NPS-SARAM--071001; ISBN 978-87-7893-225-9; Contract NKS-R-ExCoolSE; Also available at http://www.risoe.dk/rispubl/NKS/nks-160.pdf ; 4 tabs., 38 ill., 95 refs.;	Denmark	2008
36	40048133	In-Vessel Retention via External Reactor Cooling	Bachrata, Andrea; CTU in Prague, Faculty of nuclear sciences and physical engineering, V Holesovickach 2 180 00, Prague 8 (Czech republic); International Youth Nuclear Congress - IYNC (Country Unknown)	2008; 9 p.; IYNC 2008: International Youth Nuclear Congress 2008; Interlaken (Switzerland); 21-26 Sep 2008; INIS-CH--09-IYNC-2008-225; Country of input: France; 9 refs.;	Switzerland	2008

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37	42016971	Direct containment heating experiments for pressurized water reactors	Meyer, L; Albrecht, G; Kirstahler, M; Forschungszentrum Karlsruhe GmbH, Karlsruhe (Germany) Inst fuer Kern- und Energietechnik	Annual meeting on nuclear technology 2008. Proceedings; 2008; p. 228-233; 2008 annual meeting on nuclear technology; Hamburg (Germany); 27-29 May 2008;	Germany	2008
38	39121528	Evaluation of In-Vessel Corium Retention under a Severe Accident	Park, Rae-Joon; Kang, Kyung-Ho; Ha, Kwang-Soon; Kim, Jong-Tae; Koo, Kil-Mo; Cho, Young-Ro; Hong, Seong-Wan; Kim, Sang-Baik; Kim, Hee-Dong; Korea Atomic Energy Research Institute, Daejeon (Korea, Republic of)	Feb 2008; 104 p.; KAERI/TR--3537/2008; Also available from KAERI; 59 refs, 32 figs, 5 tabs;	Korea, Republic of	2008
39	42016970	The core catcher From research and development towards realization	Dyllong, Norbert; Steinwarz, Wolfgang; Siempelkamp Nukleartechnik GmbH (Germany)	Annual meeting on nuclear technology 2008. Proceedings; 2008; p. 234-237; 2008 annual meeting on nuclear technology; Hamburg (Germany); 27-29 May 2008;	Germany	2008
40	42016972	Application of the effective convectivity model to a PWR in-vessel-retention scenario	Willschuetz, HG; EON Kernkraft GmbH, Hannover (Germany); Altstadt, E; Abendroth, M; Weiss, FP; Forschungszentrum Dresden-Rossendorf eV, Dresden (Germany)	Annual meeting on nuclear technology 2008. Proceedings; 2008; p. 221-227; 2008 annual meeting on nuclear technology; Hamburg (Germany); 27-29 May 2008;	Germany	2008
41	39087305	Coolability analysis of bottom-fed debris beds in a LWR severe accident	Ma, Weimin; Dinh, Truc-Nam ; ma@safetysickthse; Royal Inst of Technology (KTH), Stockholm (Sweden)	Japan Society of Mechanical Engineers, Tokyo (Japan); Japan Society of Mechanical Engineers; Tokyo (Japan); Proceedings of the ICONE-15 (Revised). The 15th international conference on nuclear engineering; [3174 p.]; 2007; [8 p.]; ICONE-15: 15. international conference on nuclear engineering; Nagoya, Aichi (Japan); 22-26 Apr 2007; Available from Japan Society of Mechanical Engineers, 35 Shinanomachi, Shinjuku-ku, Tokyo 160-0016, Japan; This CD-ROM can be used for WINDOWS 9x/NT/2000/ME/XP, MACINTOSH; Acrobat Reader is included; Data in PDF format, Folder Name Final Paper, PaperID ICONE15-10472.pdf; 22 refs., 12 figs., 1 tab.;	Japan	2007

No.	登録番号	標題	著者情報	情報源	出版地	発行年
42	38090104	Consequences of material effects on in-vessel retention	Seiler, Jean Marie; CEA Grenoble, DTN/SE2T/LPTM, 17 rue des Martyrs, 38054 Grenoble Cedex 9 (France); Tourniaire, Bruno ; brunotourniaire@ceafr; CEA Grenoble, DTN/SE2T/LPTM, 17 rue des Martyrs, 38054 Grenoble Cedex 9 (France); Defoort, Françoise; CEA Grenoble, DTN/SE2T/LPTM, 17 rue des Martyrs, 38054 Grenoble Cedex 9 (France); Froment, Karine; CEA Grenoble, DTN/SE2T/LPTM, 17 rue des Martyrs, 38054 Grenoble Cedex 9 (France)	Nuclear Engineering and Design; Sep 2007; p. 1752-1758; NURETH-11: 11. international topical meeting on nuclear reactor thermal hydraulics; Avignon (France); 2-6 Oct 2005; v. 237(15-17); 10.1016/j.nucengdes.2007.03.007; S0029-5493(07)00205-1; Copyright (c) 2007 Elsevier Science B.V., Amsterdam, The Netherlands, All rights reserved.;	Netherlands	2007
43	39048664	Solving porous media flow for LWR components	Jaakko, Mieltinen; Mikko, Ilvonen ; jaakkomiettinen@vtffi; VTT (FI)	Japan Society of Mechanical Engineers, Tokyo (Japan); Japan Society of Mechanical Engineers; Tokyo (Japan); Proceedings of the ICONE-15 (Revised). The 15th international conference on nuclear engineering; [3174 p.]; 2007; [9 p.]; ICONE-15: 15. international conference on nuclear engineering; Nagoya, Aichi (Japan); 22-26 Apr 2007; Available from Japan Society of Mechanical Engineers, 35 Shinanomachi, Shinjuku-ku, Tokyo 160-0016, Japan; This CD-ROM can be used for WINDOWS 9x/NT/2000/ME/XP, MACINTOSH; Acrobat Reader is included; Data in PDF format, Folder Name Final Paper, PaperID ICONE15-10291.pdf; 8 refs., 7 figs., 2 tabs.;	Japan	2007
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220	24027136	Core-concrete interactions with overlying water pools	Copus, ER; Sandia National Labs, Albuquerque, NM (United States)	Proceedings of the US Nuclear Regulatory Commission nineteenth water reactor safety information meeting. Volume 2, Severe accident research; Severe accident policy implementation; Accident management; Apr 1992.; p. 235-239.; 19. Nuclear Regulatory Commission (NRC) water reactor safety information meeting.; Bethesda, MD (United States).; 28-30 Oct 1991.; NUREG/CP--0119-Vol.2.; CONF-911079--Vol.2.; OSTI as TI92014250; NTIS; INIS; GPO.;	United States	1992

No.	登録番号	標題	著者情報	情報源	出版地	発行年
221	24027225	Design bases and severe accident considerations for the System 80+ trademark containment design	Schneider, RE; Gerdes, LD; ABB-Combustion Engineering, Inc, Windson, CT (United States); Oswald, JT; Snipes, JF Jr; Duke Engineering and Services, Inc, Charlotte, NC (United States)	Proceedings of the fifth workshop on containment integrity; Jul 1992.; p. 163-178.; 5. workshop on containment integrity for nuclear power plants.; Washington, DC (United States).; 12-14 May 1992.; NUREG/CP--0120.; SAND--92-0173; CONF-920541--.; OSTI as TI92019298; NTIS; INIS; GPO.;	United States	1992
222	24027226	LIRA: An advanced containment system to minimize the accidental radioactivity releases	Turricchia, A; ENEL-DCO, Roma (Italy)	Proceedings of the fifth workshop on containment integrity; Jul 1992.; p. 247-269.; 5. workshop on containment integrity for nuclear power plants.; Washington, DC (United States).; 12-14 May 1992.; NUREG/CP--0120.; SAND--92-0173; CONF-920541--.; OSTI as TI92019298; NTIS; INIS; GPO.;	United States	1992
223	24030190	Transactions of the Twentieth Water Reactor Safety Information Meeting	Weiss, AJ; Nuclear Regulatory Commission, Washington, DC (United States) Office of Nuclear Regulatory Research; Nuclear Regulatory Commission, Washington, DC (United States)	Oct 1992.; 228 p.; 20. water reactor safety information meeting.; Bethesda, MD (United States).; 21-23 Oct 1992.; NUREG/CP--0125.; CONF-921007--.; OSTI as TI93001402; NTIS; INIS; GPO.;	United States	1992
224	24033486	Contain code applications for an improved containment concept of future light water reactors	Henneges, G; Scholtyssek, W	Research work for improving LWR safety. 1991 annual report; Aug 1992.; p. 240-248.; KFK--5050.;	Germany	1992
225	25034326	Stress analysis and scaling studies of corium crusts	Feng, Z; Mechanical and Nuclear Engineering Dept, Univ of Wisconsin, Madison, WI (United States); Engelstad, RL; Mechanical and Nuclear Engineering Dept, Univ of Wisconsin, Madison, WI (United States); Lovell, EG; Mechanical and Nuclear Engineering Dept, Univ of Wisconsin, Madison, WI (United States); Corradini, ML; Mechanical and Nuclear Engineering Dept, Univ of Wisconsin, Madison, WI (United States)	Alsmeyer, H.; Kernforschungszentrum Karlsruhe GmbH (Germany). Inst. fuer Angewandte Thermo-und Fluidodynamik (IATF); Kernforschungszentrum Karlsruhe GmbH (Germany). Projekt Nukleare Sicherheitsforschung; Nuclear Energy Agency, 75 - Paris (France).; Second OECD (NEA) CSNI specialist meeting on molten core debris-concrete interactions; 603 p.; Nov 1992.; p. 419-433.; 2. OECD (NEA) CSNI specialist meeting on molten core debris-concrete interactions.; Karlsruhe (Germany).; 1-3 Apr 1992.; KFK--5108.;	Germany	1992

No.	登録番号	標題	著者情報	情報源	出版地	発行年
226	25034330	ACE program phase D: melt attack and coolability experiments (MACE) program	Sehgal, BR; Electric Power Research Inst, Palo Alto, CA (United States); Spencer, BW; Argonne National Lab, IL (United States)	Alsmeyer, H.; Kernforschungszentrum Karlsruhe GmbH (Germany). Inst. fuer Angewandte Thermo-und Fluidodynamik (IATF); Kernforschungszentrum Karlsruhe GmbH (Germany). Projekt Nukleare Sicherheitsforschung; Nuclear Energy Agency, 75 - Paris (France).; Second OECD (NEA) CSNI specialist meeting on molten core debris-concrete interactions; 603 p.; Nov 1992.; p. 345-356.; 2. OECD (NEA) CSNI specialist meeting on molten core debris-concrete interactions.; Karlsruhe (Germany).; 1-3 Apr 1992.; KFK--5108.;	Germany	1992
227	25034453	Coolability of corium spread onto concrete under water, the PERCOLA model	Bonnet, JM; Centre d'Etudes Nucleaires, Service de Thermohydraulique des Reacteurs, 38 - Grenoble (France); Seiler, JM; Centre d'Etudes Nucleaires, Service de Thermohydraulique des Reacteurs, 38 - Grenoble (France)	Alsmeyer, H.; Kernforschungszentrum Karlsruhe GmbH (Germany). Inst. fuer Angewandte Thermo-und Fluidodynamik (IATF); Kernforschungszentrum Karlsruhe GmbH (Germany). Projekt Nukleare Sicherheitsforschung; Nuclear Energy Agency, 75 - Paris (France).; Second OECD (NEA) CSNI specialist meeting on molten core debris-concrete interactions; 603 p.; Nov 1992.; p. 473-487.; 2. OECD (NEA) CSNI specialist meeting on molten core debris-concrete interactions.; Karlsruhe (Germany).; 1-3 Apr 1992.; KFK--5108.;	Germany	1992
228	23075715	Kinetic limitations to adiabatic equilibrium models for direct containment heating (DCH)	Pilch, MM; Allen, MD; Griffith, RO; Sandia National Labs, Albuquerque, NM (United States); Nuclear Regulatory Commission, Washington, DC (United States)	[1992].; 11 p.; American Society of Mechanical Engineers national heat transfer conference and exposition.; San Diego, CA (United States).; 9-12 Aug 1992.; SAND--92-0009C.; CONF-920804--12.; Contract AC04-76DP00789.; OSTI as DE92015038; NTIS; INIS; US Govt. Printing Office Dep.;	United States	1992
229	23011947	Transactions of the nineteenth water reactor safety information meeting	Weiss, AJ; Nuclear Regulatory Commission, Washington, DC (United States) Office of Nuclear Regulatory Research; Nuclear Regulatory Commission, Washington, DC (United States)	Oct 1991.; 220 p.; 19. Nuclear Regulatory Commission (NRC) water reactor safety information meeting.; Bethesda, MD (United States).; 28-30 Oct 1991.; NUREG/CP--0118.; CONF-911079--.; OSTI as TI92001870; NTIS; INIS; GPO.;	United States	1991
230	23008764	Assessment of uncertainties in severe accident management strategies	Kastenberg, WE; Apostolakis, G; Dhir, VK; Okrent, D; Univ of California, Los Angeles (USA)	Eighteenth water reactor safety information meeting. Volume 2, Severe accident research, accident management, probabilistic risk assessment topics, individual plant examination program and other issues; Apr 1991.; p. 403-409.; 18. water reactor safety information meeting.; Rockville, MD (United States).; 22-24 Oct 1990.; NUREG/CP--0114-Vol.2.; CONF-9010185--Vol.2.; OSTI as TI91011738; NTIS; INIS; GPO.;	United States	1991

No.	登録番号	標題	著者情報	情報源	出版地	発行年
231	22051952	CSNI workshop on PSA applications and limitations	Molina, T; Nuclear Regulatory Commission, Washington, DC (USA) Div of Systems Research; Sandia National Labs, Albuquerque, NM (USA); Nuclear Regulatory Commission, Washington, DC (USA)	Feb 1991.; 475 p.; Committee on the Safety of Nuclear Installations (CSNI) on Probabilistic Safety Assessment (PSA) applications and limitations.; Santa Fe, NM (USA).; 4-6 Sep 1990.; NUREG/CP--0115.; SAND--90-2797; CONF-9009346--.; Contract AC04-76DP00789.; OSTI as TI91008351; NTIS; INIS.;	United States	1991
232	23059537	DCH experiments in the CWTI facility	Sharon, A; Binder, JL; McUumber, LM; Baumgarten, I; Spencer, BW; Argonne National Lab, IL (United States)	Transactions of the nineteenth water reactor safety information meeting; Oct 1991.; p. 4.5.; 19. Nuclear Regulatory Commission (NRC) water reactor safety information meeting.; Bethesda, MD (United States).; 28-30 Oct 1991.; NUREG/CP--0118.; CONF-911079--.; OSTI as TI92001870; NTIS; INIS; GPO.;	United States	1991
233	24070131	Cooling device for a meltdown reactor core and for the concrete structure base	Py, JP; Malaval, C; Societe Franco-Americaine de Constructions Atomiques (FRAMATOME), 92 - Paris-La-Defense (France)	26 Mar 1993; 20 Sep 1991.; 23 p.; FR patent document 2681718/A/.; FR patent application 9111654.; Available from Institut National de la Propriete Industrielle, Paris (France).; Application date: 20 Sep 1991.;	France	1991
234	22056071	Pressurized melt ejection into water pools	Tarbell, WW; Pilch, M; Sandia National Labs, Albuquerque, NM (USA); Ross, JW; Oliver, MS; Gilbert, DW; Nichols, RT; Ktech Corp, Albuquerque, NM (USA); Nuclear Regulatory Commission, Washington, DC (USA) Div of Systems Research; Sandia National Labs, Albuquerque, NM (USA); Ktech Corp, Albuquerque, NM (USA); Nuclear Regulatory Commission, Washington, DC (USA)	Mar 1991.; 97 p.; NUREG/CR--3916.; SAND--84-1531.; Contract AC04-76DP00789.; OSTI as TI91009409; NTIS; INIS; GPO.;	United States	1991
235	22063847	A systematic process for developing and assessing accident management plans	Hanson, DJ; Blackman, HS; Meyer, OR; Ward, LW; EG and G Idaho, Inc, Idaho Falls, ID (USA); Nuclear Regulatory Commission, Washington, DC (USA) Div of Systems Research; EG and G Idaho, Inc, Idaho Falls, ID (USA); Nuclear Regulatory Commission, Washington, DC (USA)	Apr 1991.; 91 p.; NUREG/CR--5543.; EGG--2595.; Contract AC07-76ID01570.; OSTI as TI91010970; NTIS; INIS; GPO.;	United States	1991

No.	登録番号	標題	著者情報	情報源	出版地	発行年
236	22077531	Impact of the filtered venting system design upon the total radioactive release in case of a severe accident and a comparison of European requirements	Cederqvist, H; Elisson, K; ABB Atom, Vaesteras (Sweden); Loewenhielm, G; Appelgren, E; Swedish State Power Board, Vaellingby (Sweden)	Proceeding of the 21st DOE/NRC nuclear air cleaning conference. Volume 2, Sessions 9--16; Feb 1991.; p. 933-945.; 21. Department of Energy/Nuclear Regulatory Commission (DOE/NRC) nuclear air cleaning conference.; San Diego, CA (USA).; 13-16 Aug 1990.; NUREG/CP--0116-Vol.2.; CONF-900809--Vol.2.; OSTI as TI91009673; NTIS; INIS; GPO.;	United States	1991
237	22085550	Identification and initial assessment of candidate BWR late-phase in-vessel accident management strategies	Hodge, SA; Oak Ridge National Lab, TN (USA); Nuclear Regulatory Commission, Washington, DC (USA)	15 Apr 1991.; 32 p.; United States Nuclear Regulatory Commission (USNRC) BMU/GRS (Society for Reactor Safety) accident management information exchange meeting.; Rockville, MD (USA).; 15 Apr 1991.; CONF-9104223--1.; Contract AC05-84OR21400.; OSTI as DE91010997; NTIS; INIS; US Govt. Printing Office Dep.;	United States	1991
238	23008762	MELCOR analysis of the TMI-2 accident	Boucheron, EA; Sandia National Laboratories, Albuquerque, NM (USA)	Eighteenth water reactor safety information meeting. Volume 2, Severe accident research, accident management, probabilistic risk assessment topics, individual plant examination program and other issues; Apr 1991.; p. 207-230.; 18. water reactor safety information meeting.; Rockville, MD (United States).; 22-24 Oct 1990.; NUREG/CP--0114-Vol.2.; CONF-9010185--Vol.2.; OSTI as TI91011738; NTIS; INIS; GPO.;	United States	1991
239	23036656	Cooling of core debris within the reactor vessel lower head	Henry, RE; Burelbach, JP; Hammersley, RJ; Henry, CE; Fauske and Associates, Inc, Burr Ridge, IL (United States); Klopp, GT; Commonwealth Edison Co, Downers Grove, IL (United States)	Transactions of the American Nuclear Society.; (1991).; Annual meeting of the American Nuclear Society (ANS).; Orlando, FL (United States).; 2-6 Jun 1991.; v. 63 p. 260-261.; CONF-910603--;	United States	1991
240	23036778	Nuclear plant analyzer applied to fuel rod damage	Snider, DM; Wagner, KL; Grush, WH; Idaho National Engineering Lab, Idaho Falls (United States)	Transactions of the American Nuclear Society.; (1991).; Annual meeting of the American Nuclear Society (ANS).; Orlando, FL (United States).; 2-6 Jun 1991.; v. 63 p. 325-327.; CONF-910603--;	United States	1991
241	23059421	Planned MELCOR improvements and assessment	Summers, RM; Kmetyk, LN; Sandia National Lab, Albuquerque, NM (United States)	Transactions of the nineteenth water reactor safety information meeting; Oct 1991.; p. 1.5-1.6.; 19. Nuclear Regulatory Commission (NRC) water reactor safety information meeting.; Bethesda, MD (United States).; 28-30 Oct 1991.; NUREG/CP--0118.; CONF-911079--.; OSTI as TI92001870; NTIS; INIS; GPO.;	United States	1991

No.	登録番号	標題	著者情報	情報源	出版地	発行年
242	23059469	SCDAP/RELAP5/MOD3 code development and assessment	Allison, CM; Heath, CH; Siefken, LJ; Hohorst, JK; Idaho National Engineering Lab, Idaho Falls (United States)	Transactions of the nineteenth water reactor safety information meeting; Oct 1991.; p. 12.11-12.12.; 19. Nuclear Regulatory Commission (NRC) water reactor safety information meeting.; Bethesda, MD (United States).; 28-30 Oct 1991.; NUREG/CP--0118.; CONF-911079--.; OSTI as TI92001870; NTIS; INIS; GPO.;	United States	1991
243	23059473	ACE program Phases C and D: Molten corium concrete interaction (MCCI) experiments and corium melt coolability experiments (MACE)	Sehgal, BR; Electric Power Research Inst, Palo Alto, CA (United States); Spencer, BW; Thompson, DH; Fink, JK; Farmer, MT; Argonne National Lab, IL (United States)	Transactions of the nineteenth water reactor safety information meeting; Oct 1991.; p. 13.7-13.8.; 19. Nuclear Regulatory Commission (NRC) water reactor safety information meeting.; Bethesda, MD (United States).; 28-30 Oct 1991.; NUREG/CP--0118.; CONF-911079--.; OSTI as TI92001870; NTIS; INIS; GPO.;	United States	1991
244	22056105	Estimation of containment pressure loading due to direct containment heating for the Zion Plant	Tutu, NK; Ginsberg, T; Brookhaven National Lab, Upton, NY (USA); Park, CK; Korea Atomic Energy Research Inst, Daeduk (Republic of Korea); Grimshaw, CA; Margrove Consulting Ltd, London (UK); Nuclear Regulatory Commission, Washington, DC (USA) Div of Systems Research; Brookhaven National Lab, Upton, NY (USA); Nuclear Regulatory Commission, Washington, DC (USA)	Mar 1991.; 51 p.; NUREG/CR--5282.; BNL-NUREG--52181.; Contract AC02-76CH00016.; OSTI as TI91009157; NTIS; INIS; GPO.;	United States	1991
245	23008686	Comparison of analytical models and calculated results of source term evaluation codes	Kondo, Shunsuke; Univ of Tokyo (Japan); Abe, Kiyoharu; JAERI, Ibaraki (Japan)	CSNI workshop on PSA applications and limitations. Proceedings; Feb 1991.; p. 273-287.; Committee on the Safety of Nuclear Installations (CSNI) on Probabilistic Safety Assessment (PSA) applications and limitations.; Santa Fe, NM (United States).; 4-6 Sep 1990.; NUREG/CP--0115.; SAND--90-2797; CONF-9009346--.; OSTI as TI91008351; NTIS; INIS.;	United States	1991
246	23008699	Safety goals and functional performance criteria	Youngblood, RW; Pratt, WT; Brookhaven National Lab, Upton, NY (USA); Hardin, WB; Nuclear Regulatory Commission, Washington, DC (USA)	CSNI workshop on PSA applications and limitations. Proceedings; Feb 1991.; p. 413-429.; Committee on the Safety of Nuclear Installations (CSNI) on Probabilistic Safety Assessment (PSA) applications and limitations.; Santa Fe, NM (United States).; 4-6 Sep 1990.; NUREG/CP--0115.; SAND--90-2797; CONF-9009346--.; OSTI as TI91008351; NTIS; INIS.;	United States	1991

No.	登録番号	標題	著者情報	情報源	出版地	発行年
247	23017315	Experimental results of direct containment heating by high-pressure melt ejection into the Surtsey vessel: The DCH-3 and DCH-4 tests	Allen, MD; Pilch, M; Brockmann, JE; Tarbell, WW; Sandia National Labs, Albuquerque, NM (United States); Nichols, RT; Ktech Corp, Albuquerque, NM (United States); Sweet, DW; AEA Technology, Winfrith (United Kingdom); Sandia National Labs, Albuquerque, NM (United States); USDOE, Washington, DC (United States)	Aug 1991.; 249 p.; SAND--90-2138.; Contract AC04-76DP00789.; OSTI as DE92001670; NTIS; INIS; US Govt. Printing Office Dep.;	United States	1991
248	23059543	Cooling of core debris within the reactor vessel lower head	Henry, RE; Burelbach, JP; Hammersley, RJ; Henry, CE; Fauske and Associates, Inc, Burr Ridge, IL (United States); Klopp, GT; Commonwealth Edison Co, Downers Grove, IL (United States)	Transactions of the nineteenth water reactor safety information meeting; Oct 1991.; p. 12.3-12.4.; 19. Nuclear Regulatory Commission (NRC) water reactor safety information meeting.; Bethesda, MD (United States).; 28-30 Oct 1991.; NUREG/CP--0118.; CONF-911079--.; OSTI as TI92001870; NTIS; INIS; GPO.;	United States	1991
249	23059552	Primary system thermal-hydraulics during the core uncover phase of the TMI-2 accident	Bessette, D; Nuclear Regulatory Commission, Washington, DC (United States)	Transactions of the nineteenth water reactor safety information meeting; Oct 1991.; p. 18.7-18.8.; 19. Nuclear Regulatory Commission (NRC) water reactor safety information meeting.; Bethesda, MD (United States).; 28-30 Oct 1991.; NUREG/CP--0118.; CONF-911079--.; OSTI as TI92001870; NTIS; INIS; GPO.;	United States	1991
250	24015743	Joint severe accident studies under US-USSR JCCCNRS	Asmolov, V; Kurchatov Institute of Atomic Energy, Moscow (Russian Federation)	Transactions of the American Nuclear Society.; (1991).; p. 406.; 1991 Winter meeting of the American Nuclear Society (ANS) session on fundamentals of fusion reactor thermal hydraulics.; San Francisco, CA (United States).; 10-15 Nov 1991.; v. 63.; CONF-911107--.;	United States	1991
251	24015896	Passive containment system for advanced light water plants	Kleimola, FW; Falls, OB Jr	Transactions of the American Nuclear Society.; (1991).; p. 413-415.; 1991 Winter meeting of the American Nuclear Society (ANS) session on fundamentals of fusion reactor thermal hydraulics.; San Francisco, CA (United States).; 10-15 Nov 1991.; v. 63.; CONF-911107--.;	United States	1991
252	25022670	Nuclear reactor with recuperating and cooling device of a meltdown core	Jude, P; Societe Franco-Americaine de Constructions Atomiques (FRAMATOME), 92 - Paris-La-Defense (France)	11 Jun 1993; 4 Dec 1991.; 20 p.; FR patent document 2684790/A/.; FR patent application 9115028.; Available from Institut National de la Propriete Industrielle, Paris (France).; Application date: 4 Dec 1991.;	France	1991

No.	登録番号	標題	著者情報	情報源	出版地	発行年
253	23024833	Fission product chemistry in severe nuclear reactor accidents, specialists' meeting at JRC-Ispra, 15-17 January 1990	Nichols, AL; AEA Reactor Services, Winfrith (United Kingdom)	May 1990.; 34 p.; Fission product chemistry in severe nuclear reactor accidents, specialists' meeting.; Ispra (Italy).; 15-17 Jan 1990.; AEA-TRS--5024.;	United Kingdom	1990
254	21036793	Vaporization of strontium, barium, lanthanum, and uranium from mixtures of urania, zirconia, steel, and concretes at 2150 K and 2400 K	Roche, MF; Settle, JL; Leibowitz, L; Johnson, CE; Argonne National Lab, IL (USA); Electric Power Research Inst, Palo Alto, CA (USA); Argonne National Lab, IL (USA)	Jan 1990.; 86 p.; EPRI-NP--6613.; Contract W-31109-ENG-38.; Research Reports Center, Box 50490, Palo Alto, CA 94303.;	United States	1990
255	21054098	Accident management information needs	Hanson, DJ; Ward, LW; Nelson, WR; Meyer, OR; EG and G Idaho, Inc, Idaho Falls, ID (USA); Nuclear Regulatory Commission, Washington, DC (USA) Div of Systems Research; EG and G Idaho, Inc, Idaho Falls, ID (USA)	Apr 1990.; 67 p.; NUREG/CR--5513-Vol.1.; EGG--2592-Vol.1.; Contract AC07-76ID01570.; NTIS, PC A04/MF A01 - GPO as T190009441; OSTI; INIS.;	United States	1990
256	22063806	ACE program Phase C: Molten corium concrete interaction (MCCI) and corium melt coolability experiments	Sehgal, BR; Spencer, BW	Transactions of the eighteenth water reactor safety information meeting; Oct 1990.; p. 13.11-13.12.; NUREG/CP--0113.; OSTI as T191000893; GPO.;	United States	1990
257	21043161	Melt progression, oxidation, and natural convection in a severely damaged reactor core	Dosanjh, SS; Sandia National Labs, Albuquerque, NM (USA); Nuclear Regulatory Commission, Washington, DC (USA) Div of Systems Research; Sandia National Labs, Albuquerque, NM (USA)	Feb 1990.; 87 p.; NUREG/CR--5316.; SAND--88-3476.; Contract AC04-76DP00789.; NTIS, PC A05/MF A01 - GPO as T190007218; OSTI; INIS.;	United States	1990
258	21076048	Technical support for the EPRI [Electric Power Research Institute] debris coolability requirement for advanced light water reactors	EG and G Idaho, Inc, Idaho Falls, ID (USA); Fauske and Associates, Inc, Burr Ridge, IL (USA)	Mar 1990.; 74 p.; DOE/ID--10278.; MISC--90057.; Contract AC07-76ID01570.; NTIS, PC A04/MF A01 as DE90013095; OSTI; INIS; US Govt. Printing Office Dep.;	United States	1990

No.	登録番号	標題	著者情報	情報源	出版地	発行年
259	21047045	Damaged Fuel Experiment DF-1	Gasser, RD; Fryer, CP; Gauntt, RO; Marshall, AC; Reil, KO; Stalker, KT; Nuclear Regulatory Commission, Washington, DC (USA) Div of Systems Research; Sandia National Labs, Albuquerque, NM (USA)	Jan 1990.; 261 p.; NUREG/CR--4668.; SAND--86-1030.; Contract AC04-76DP00789.; NTIS, PC E10/MF E01 as TI90007705; OSTI; INIS.; Includes 2 microfiche supplements.;	United States	1990
260	22060718	Radionuclide transport and retention in reactor cooling systems during sever accidents	Wassel, AT; Farr, JL Jr; Bugby, DC; Merilo, M	Transactions of the American Nuclear Society.; (1990).; American Nuclear Society (ANS) winter meeting.; Washington, DC (USA).; 11-15 Nov 1990.; v. 62 p. 699-700.; CONF-901101--.;	United States	1990
261	23005620	Mitigation of direct containment heating by intentional reactor coolant system depressurization	Golden, DW; Hallbert, BP; Miller, JD; Idaho National Engineering Lab, Idaho Falls (USA); Odar, F; Nuclear Regulatory Commission, Rockville, MD (USA)	Seventeenth water reactor safety information meeting: Proceedings. Volume 2, Accident management; severe accident research; earth sciences; probabilistic risk assessment; seismic and structural engineering; Mar 1990.; p. 67-90.; 17. water reactor safety information meeting.; Rockville, MD (United States).; 23-25 Oct 1989.; NUREG/CP--0105-Vol.2.; OSTI as TI90008996; INIS.;	United States	1990
262	27066120	Containment as the focus of US NRC's revised severe accident research program	Theofanous, TG; California Univ, Santa Barbara, CA (United States); Speis, T; Sheron, B; Eltawila, F; Nuclear Regulatory Commission, Washington, DC (United States)	Lawrence, S.R.; Canadian Nuclear Society, Toronto, ON (Canada); International Atomic Energy Agency, Vienna (Austria); Ontario Hydro, Toronto (Canada). CANDU Owners Group (COG); European Nuclear Society (ENS), Bern (Switzerland); Atomic Energy Society of Japan, Tokyo (Japan); American Nuclear Society, Chicago, IL (United States).; Proceedings of the 2. international conference on containment design and operation. Vol. 1,2; [1326 p.]; 1990.; [13 p.]; 2. International conference on containment design and operation.; Toronto, ON (Canada).; 14-17 Oct 1990.; INIS-mf--14838.;	Canada	1990
263	22063749	Heat transfer phenomena relevant to severe accidents	Dallman, RJ; Duffey, RB; EG and G Idaho, Inc, Idaho Falls, ID (USA); USDOE, Washington, DC (USA)	[1990].; 14 p.; 9. international heat transfer conference.; Jerusalem (Israel).; 19-24 Aug 1990.; EGG-M--90119.; CONF-900803--4.; Contract AC07-76ID01570.; OSTI as DE91010087; NTIS; INIS; US Govt. Printing Office Dep.;	United States	1990

No.	登録番号	標題	著者情報	情報源	出版地	発行年
264	22026371	Metal-fueled HWR [heavy water reactors] severe accident issues: Differences and similarities to commercial LWRs [light water reactors]	Ellison, PG; Hyder, ML; Monson, PR; Westinghouse Savannah River Co, Aiken, SC (USA); Coryell, EW; EG and G Idaho, Inc, Idaho Falls, ID (USA); Westinghouse Savannah River Co, Aiken, SC (USA)	[1990].; 7 p.; American Nuclear Society topical meeting on safety of non-commercial reactors and irradiation facilities.; Boise, ID (USA).; 30 Sep 1990.; WSRC-MS--90-18.; CONF-9010151--4.; Contract AC09-89SR18035.; NTIS, PC A02/MF A01 as DE91004255; OSTI; INIS; US Govt. Printing Office Dep.;	United States	1990
265	22032017	Safety goals and functional performance criteria	Youngblood, RW; Pratt, WT; Brookhaven National Lab, Upton, NY (USA); Hardin, WB; Nuclear Regulatory Commission, Washington, DC (USA) Div of Regulatory Applications; Brookhaven National Lab, Upton, NY (USA)	[1990].; 17 p.; Committee on Safety of Nuclear Installations (CSNI).; Santa Fe, NM (USA).; 4-6 Sep 1990.; BNL-NUREG--44451.; CONF-9009226--4.; Contract AC02-76CH00016.; NTIS, PC A03/MF A01 as DE91004751; OSTI; INIS; US Govt. Printing Office Dep.;	United States	1990
266	20069456	MELPROG-POW/MOD1: A two-dimensional, mechanistic code for analysis of reactor core melt progression and vessel attack under severe accident conditions	Dosanjh, SS; Nuclear Regulatory Commission, Washington, DC (USA) Div of Systems Research; Sandia National Labs, Albuquerque, NM (USA)	May 1989.; 391 p.; NUREG/CR--5193.; SAND--88-1824.; NTIS, PC A17/MF A01 - US Govt. Printing Office. - OSTI as TI89012938.;	United States	1989
267	20079227	Integrated MELPROG/TRAC analyses of a PWR station blackout	Heames, TJ; Smith, RC; Science Applications International Corp, Albuquerque, NM (USA); Sandia National Labs, Albuquerque, NM (USA); Sandia National Labs, Albuquerque, NM (USA)	1989.; 11 p.; ASME/AIChE national heat transfer conference.; Philadelphia, PA (USA).; 6-9 Aug 1989.; SAND--88-7158C.; CONF-890819--10.; Contract DOE AC04-76DP00789.; Available from NTIS, PC A03/MF A01 - OSTI as DE89010434.;	United States	1989
268	21019325	DOE modifications to the MAAP [Modular Accident Analysis Program] code	Plys, MG; Wu, Chun-Der; Epstein, M; Hammersley, RJ; Fauske and Associates, Inc, Burr Ridge, IL (USA); EG and G Idaho, Inc, Idaho Falls, ID (USA); Fauske and Associates, Inc, Burr Ridge, IL (USA)	Aug 1989.; 115 p.; DOE/ID--10216-Vol.6.; Contract AC07-76ID01570.; Available from NTIS, PC A06/MF A01 as DE90002006; OSTI; INIS; US Govt. Printing Office Dep.;	United States	1989
269	21010343	Uncertainties in hydrogen combustion for nuclear reactor safety	Stamps, DW; Worthington, PR; Wong, CC; Sandia National Labs, Albuquerque, NM (USA); Nuclear Regulatory Commission, Washington, DC (USA); Sandia National Labs, Albuquerque, NM (USA); Sandia National Labs, Albuquerque, NM (USA)	1989.; 29 p.; 3. international seminar on containment of nuclear reactors.; Los Angeles, CA (USA).; 10-11 Aug 1989.; SAND--89-1854C.; CONF-8908127--1.; Contract AC04-76DP00789.; Available from NTIS, PC A03/MF A01 as DE89016030; OSTI; INIS; US Govt. Printing Office Dep.;	United States	1989

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270	22012600	A sensitivity analysis of direct containment heating	Park, CK; Tutu, NK; Grimshaw, CA; Ginsberg, T; Brookhaven National Lab, Upton, NY (USA)	Transactions of the American Nuclear Society.; (1989).; Winter meeting of the American Nuclear Society (ANS) and nuclear power and technology exhibit.; San Francisco, CA (USA).; 26-30 Nov 1989.; v. 60 p. 429-431.; CONF-891103--;	United States	1989
271	22052020	Mitigation of direct containment heating by intentional reactor coolant system depressurization	Golden, DW; Miller, JD; Idaho National Engineering Lab, Idaho Falls (USA); Odar, F; Nuclear Regulatory Commission, Rockville, MD (USA)	Transactions of the seventeenth water reactor safety information meeting; Oct 1989.; p. 1.11-1.13.; 17. water reactor safety information meeting.; Rockville, MD (USA).; 23-25 Oct 1989.; NUREG/CP--0104.; CONF-8910222--Summs.; NTIS, PC A09/MF A01 as DE90000746.;	United States	1989
272	20037728	Safety research programs sponsored by Office of Nuclear Regulatory Research: Progress report, July 1--September 30, 1988	Weiss, AJ; Nuclear Regulatory Commission, Washington, DC (USA) Office of Nuclear Regulatory Research; Brookhaven National Lab, Upton, NY (USA)	Feb 1989.; 203 p.; NUREG/CR--2331-Vol.8-No.3.; BNL-NUREG--51454-Vol.8-No.3.; NTIS, PC A09/MF A01 - US Govt. Printing Office. - OSTI as TI89007082.;	United States	1989
273	21036802	User's manual for CONTAIN 11: A computer code for severe nuclear reactor accident containment analysis	Murata, KK; Carroll, DE; Washington, KE; Gelbard, F; Valdez, GD; Williams, DC; Bergeron, KD; Sandia National Labs, Albuquerque, NM (USA); Technadyne Engineering Consultants, Inc, Albuquerque, NM (USA); Sandia National Labs, Albuquerque, NM (USA); Nuclear Regulatory Commission, Washington, DC (USA) Div of Systems Research; Sandia National Labs, Albuquerque, NM (USA)	Nov 1989.; 443 p.; NUREG/CR--5026.; SAND--87-2309.; Contract AC04-76DP00789.; NTIS, PC A19/MF A01 as TI90003993; OSTI; INIS.;	United States	1989
274	22051926	Melt Attack and Coolability Experiments (MACE) program elements and the scoping test	Sehgal, BR; Rahn, FJ; Electric Power Research Institute, Palo Alto, CA (USA); Spencer, BW; Argonne National Lab, IL (USA)	Transactions of the seventeenth water reactor safety information meeting; Oct 1989.; p. 15.9-15.10.; 17. water reactor safety information meeting.; Rockville, MD (USA).; 23-25 Oct 1989.; NUREG/CP--0104.; CONF-8910222--Summs.; NTIS, PC A09/MF A01 as DE90000746.;	United States	1989

No.	登録番号	標題	著者情報	情報源	出版地	発行年
275	22085716	A model for recovery of a badly degraded core	Sharon, A; Henry, RE; Wu, Chunder; Hammersley, RJ; Fauske ampersand Associates, Inc, Burr Ridge, IL (USA)	Transactions of the American Nuclear Society.; (Nov 1989).; Winter meeting of the American Nuclear Society (ANS) and nuclear power and technology exhibit.; San Francisco, CA (USA).; 26-30 Nov 1989.; v. 60 p. 747-748.; CONF-891103--;	United States	1989
276	21036044	Safety research programs sponsored by Office of Nuclear Regulatory Research	Weiss, AJ; Azarm, A; Baum, JW; Boccio, JL; Carew, J; Diamond, DJ; Fitzpatrick, R; Ginsberg, T; Greene, GA; Guppy, JG; Haber, SB; Hall, RE; Higgins, JC; Khan, TA; Lehner, JR; Madni, IK; Meinhd, C; Philippacoupoulos, AJ; Pratt, WT; Ruger, C; Rohatgi, US; Samanta, P; Taylor, JH; Tutu, N; Van Tuyle, GJ; Wulff, W; Youngblood, R; Nuclear Regulatory Commission, Washington, DC (USA) Office of Nuclear Regulatory Research; Brookhaven National Lab, Upton, NY (USA)	Dec 1989.; 158 p.; NUREG/CR--2331-Vol.9-No.2.; BNL-NUREG--51454-Vol.9-No.2.; Contract AC02-76CH00016.; NTIS, PC A07/MF A01 - GPO - OSTI as TI90004652.;	United States	1989
277	21036799	The DF-4 fuel damage experiment in ACRR [Annual Core Research Reactor] with a BWR [Boiling Water Reactor] control blade and channel box	Gauntt, RO; Gasser, RD; Ott, LJ; Sandia National Labs, Albuquerque, NM (USA); Nuclear Regulatory Commission, Washington, DC (USA) Div of Systems Research; Sandia National Labs, Albuquerque, NM (USA)	Nov 1989.; 176 p.; NUREG/CR--4671.; SAND--86-1443.; Contract AC04-76DP00789.; NTIS, PC A08/MF A01 - GPO as TI90006723; OSTI; INIS.; This report contains 3 microfiche supplements.;	United States	1989
278	22012636	TMLB ¹ -sequence simulation for a VVER-1000-type reactor	Sdouz, G	Transactions of the American Nuclear Society.; (1989).; Winter meeting of the American Nuclear Society (ANS) and nuclear power and technology exhibit.; San Francisco, CA (USA).; 26-30 Nov 1989.; v. 60 p. 428-429.; CONF-891103--;	United States	1989
279	22085712	Effectivity of a passive heat removal system for a new VVER reactor	Kirmse, RE; Voznesensky, VA; Kurchatov Institute of Atomic Energy, Moscow (USSR)	Transactions of the American Nuclear Society.; (Nov 1989).; Winter meeting of the American Nuclear Society (ANS) and nuclear power and technology exhibit.; San Francisco, CA (USA).; 26-30 Nov 1989.; v. 60 p. 722.; CONF-891103--;	United States	1989

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280	21014125	SCDAP/RELAP5 code assessment: Models for the liquefaction of UO ₂ in molten zircaloy	Allison, CM; Carlson, ER; Cheng, TC; Siefken, LJ; Hohorst, JK; EG and G Idaho, Inc, Idaho Falls, ID (USA); EG and G Idaho, Inc, Idaho Falls, ID (USA)	[1989].; 6 p.; 4. international topical meeting on nuclear reactor thermal hydraulics.; Karlsruhe (Germany, F.R.); 10-13 Oct 1989.; EGG-M--88272.; CONF-891004--3.; Contract AC07-76ID01570.; Available from NTIS, PC A02/MF A01 as DE89016198; OSTI; INIS; US Govt. Printing Office Dep.;	United States	1989
281	21068132	A model for a recovery of a badly degraded core	Sharon, A; Henry, RE; Wu, Chun-Der; Hammersley, RJ; Fauske and Associates, Inc, Burr Ridge, IL (USA); EG and G Idaho, Inc, Idaho Falls, ID (USA); Fauske and Associates, Inc, Burr Ridge, IL (USA)	[1989].; 40 p.; Winter meeting of the American Nuclear Society.; San Francisco, CA (USA).; 26-30 Nov 1989.; EGG-M--89219.; CONF-891103--73.; Contract AC07-76ID01570.; NTIS, PC A03/MF A01 as DE90010932; OSTI; INIS; US Govt. Printing Office Dep.;	United States	1989
282	19101146	TMI-2 accident scenario development	Tolman, EL; Kuan, P; Broughton, JM; EG and G Idaho Inc, Idaho Falls (USA)	Proceedings of the US Nuclear Regulatory Commission fifteenth water reactor safety information meeting: Volume 6, Decontamination and decommissioning, accident management, TMI-2; Feb 1988.; p. 155-173.; 15. water reactor safety information meeting.; Gaithersburg, MD (USA).; 26-30 Oct 1987.; NUREG/CP--0091-Vol.6.; NTIS, PC A 12 - US Govt. Printing Office; 3 as TI88006636.;	United States	1988
283	19105013	Fission product release characteristics into containment under design basis and severe accident conditions	Nourbakhsh, HP; Khatib-Rahbar, M; Davis, RE; Nuclear Regulatory Commission, Washington, DC (USA) Office of Nuclear Regulatory Research; Brookhaven National Lab, Upton, NY (USA)	Mar 1988.; 105 p.; NUREG/CR--4881.; BNL-NUREG--52059.; NTIS, PC A06 - US Govt. Printing Office. as TI88010887.;	United States	1988
284	20019261	Direct containment heating: is the hazard real	Lee, TM	Proceedings of the US Nuclear Regulatory Commission fifteenth water reactor safety information meeting: Volume 4, Separate effects/experiments and analyses, Source term uncertainty analysis, Integral systems testing, 2D/3D research; Feb 1988.; p. 209-229.; 15. water reactor safety information meeting.; Gaithersburg, MD (USA).; 26-30 Oct 1987.; NUREG/CP--0091-Vol.4.; NTIS, PC A99/MF A01 - US Govt. Printing Office. as TI88007181.;	United States	1988
285	19078569	Proceedings of the US Nuclear Regulatory Commission fifteenth water reactor safety information meeting: Volume 5, Industry safety research, International Code Assessment Program	Weiss, AJ; Nuclear Regulatory Commission, Washington, DC (USA) Office of Nuclear Regulatory Research	Feb 1988.; 469 p.; 15. water reactor safety information meeting.; Gaithersburg, MD (USA).; 26-30 Oct 1987.; NUREG/CP--0091-Vol.5.; CONF-8710111--Vol.5.; Available from NTIS, PC A20/MF A01; 1 as TI88006492.; Portions of this document are illegible in microfiche products.;	United States	1988

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286	20084631	Experimental and theoretical investigation of melt propagation in rubble beds for application in severe-accident analyses	Ahmed, EKS; Podowski, MZ; Lahey, R Jr	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 367-368.; CONF-881011--;	United States	1988
287	20085875	Direct containment heating and aerosol generation during high-pressure-melt expulsion experiments	Tarbell, WW; Brockmann, JE; Washington, KE; Pilch, M; Marx, KD	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 361.; CONF-881011--;	United States	1988
288	21023733	Sampling and examination methods used for TMI-2 samples	Marley, AW; Akers, DW; McIsaac, CV; EG and G Idaho, Inc, Idaho Falls, ID (USA)	1988.; 32 p.; TMI-2 accident materials behavior, plant technology and recovery.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; EGG-M--88177.; CONF-881024--19.; Contract AC07-76ID01570.; NTIS, PC A03/MF A01 as DE90002000; OSTI; INIS; US Govt. Printing Office Dep.;	United States	1988
289	20019263	Results of the four PBF severe fuel damage tests	Osetek, DJ; Idaho National Engineering Lab, Idaho Falls (USA)	Proceedings of the US Nuclear Regulatory Commission fifteenth water reactor safety information meeting: Volume 4, Separate effects/experiments and analyses, Source term uncertainty analysis, Integral systems testing, 2D/3D research; Feb 1988.; p. 299-323.; 15. water reactor safety information meeting.; Gaithersburg, MD (USA).; 26-30 Oct 1987.; NUREG/CP--0091-Vol.4.; NTIS, PC A99/MF A01 - US Govt. Printing Office. as TI88007181.;	United States	1988
290	20060967	Containment and release management	Lehner, JR; Pratt, WT; Brookhaven National Lab, Upton, NY (USA)	1988.; 9 p.; 16. water reactor safety information meeting.; Gaithersburg, MD (USA).; 24-27 Oct 1988.; BNL-NUREG--42261.; CONF-8810155--47.; Available from NTIS, PC A02/MF A01 - OSTI; 1 as DE89009526.; Portions of this document are illegible in microfiche products.;	United States	1988
291	20084633	Analysis of fission product revaporization in a BWR reactor cooling system during a station blackout accident	Yang, JW; Schmidt, E; Cazzoli, E; Khatib-Rahbar, M; Brookhaven National Lab, Upton, NY (USA)	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 370-372.; CONF-881011--;	United States	1988

No.	登録番号	標題	著者情報	情報源	出版地	発行年
292	20084638	Analysis of the late core melt progression phase of severe reactor accidents using the MELPROG code	Dosanjh, SS	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 368-369.; CONF-881011--;	United States	1988
293	21002592	Disassembly and defueling of the upper core support assembly	Rodabaugh, JM	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 497-498.; CONF-881011--;	United States	1988
294	21002602	Criticality prevention during postaccident decontamination of TMI-2 [Three Mile Island Unit 2] plant systems	Palau, G L	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 509-510.; CONF-881011--;	United States	1988
295	21006444	TMI-2 [Three Mile Island Unit 2] ex-vessel defueling	Wolfgang, RJ; Patterson, RL	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 498-499.; CONF-881011--;	United States	1988
296	21014192	Considerations for severe-accident management strategies in a pressurized water reactor	Carbajo, JJ; Carter, JC; IT Corp, Oak Ridge, TN (USA)	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 159-160.; CONF-881011--;	United States	1988
297	19092525	Research on direct containment heating and pressurized melt expulsion from the reactor coolant system	Tarbell, WW; Pilch, M; Brockmann, JE; Powers, DA; Sandia National Labs, Albuquerque, NM (USA)	1988.; 19 p.; International symposium on severe accidents in nuclear power plants.; Sorrento (Italy).; 21-25 Mar 1988.; SAND--88-0717C.; CONF-880309--4; IAEA-SM--296/101.; Available from NTIS, PC A03/MF A01 as DE88008472.;	United States	1988

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298	20037006	Accident management for severe accidents	Bari, RA; Pratt, WT; Lehner, J; Leonard, M; Disalvo, R; Sheron, B; Brookhaven National Lab, Upton, NY (USA)	1988.; 8 p.; International European Nuclear Society/American Nuclear Society meeting on thermal reactor safety.; Avignon (France).; 2-7 Oct 1988.; BNL-NUREG--41964.; CONF-881014--16.; Available from NTIS, PC A02/MF A01 - OSTI; 1 as DE89004899.; Portions of this document are illegible in microfiche products.;	United States	1988
299	20061098	Summary of recent TMI-2 accident scenario development work	Golden, DW; Kuan, P; EG and G Idaho, Inc, Idaho Falls, ID (USA)	1988.; 4 p.; 16. light water reactor safety conference.; Washington, DC (USA).; 24-28 Oct 1988.; EGG-M--88324.; CONF-8810244--3.; Available from NTIS, PC A02/MF A01 - OSTI; 1 as DE89009766.; Portions of this document are illegible in microfiche products.;	United States	1988
300	21002601	Criticality analysis support for TMI-2 [Three Mile Island Unit 2] fuel removal operations	Parks, CV; Westfall, RM; Oak Ridge National Lab, TN (USA)	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 507-508.; CONF-881011--.;	United States	1988
301	21002603	MARCH calculations performed for the TMI-2 [Three Mile Island Unit] analysis exercise program	Wooton, RO	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 444.; CONF-881011--.;	United States	1988
302	21006427	Ex-vessel fuel characterization results in the reactor building	Kobayashi, R; Distenfeld, C H; Ferguson, D E	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 464.; CONF-881011--.;	United States	1988
303	21006440	Data acquisition methods used at TMI-2 [Three Mile Island Unit 2]	Patterson, RL; Estabrook, ML; Wilson, DC	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 492-493.; CONF-881011--.;	United States	1988

No.	登録番号	標題	著者情報	情報源	出版地	発行年
304	21006460	RCS characterization and SNM accountability: Trace fuel circulation in the RCS, reactor building and auxiliary building	Greenborg, J	Transactions of the American Nuclear Society.; (1988).; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; v. 57 p. 462.; CONF-881011--.;	United States	1988
305	21068140	Design considerations for severe accident containment performance	Davis, GA; Matzie, RA; Green, MD; Combustion Engineering, Inc, Windsor, CT (USA); Brewer, HD; Fox, WA III; Duke Power Company, Charlotte, NC (USA)	Fourth workshop on containment integrity. Proceedings; Nov 1988.; p. 37-46.; 4. workshop on integrity of containments for nuclear power plants.; Arlington, VA (USA).; 14-17 Jun 1988.; NUREG/CP--0095.; SAND--88-1836; CONF-880616--.; NTIS, PC A99/MF A01 - I as DE89004641.;	United States	1988
306	22029370	BWR [boiling water reactor] core criticality versus water level during an ATWS [anticipated transient without scram] event	Sehgal, BR; Electric Power Research Institute, Palo Alto, CA (USA); Peng, CM; Systems Control Inc, Palo Alto, CA (USA); Maly, J; Science Application International, Albuquerque, NM (USA)	Proceedings of the 1988 International reactor physics conference. Volume 4; 1988.; p. IV.31-IV.43.; International reactor physics conference.; Jackson Hole, WY (USA).; 18-22 Sep 1988.; American Nuclear Society.; LaGrange Park, IL (USA).; CONF-880911--Vol.4.; American Nuclear Society, 555 N. Kensington Avenue, LaGrange Park, IL 60525.;	United States	1988
307	22029672	Nuclear power reactors of new generation	Ponomarev-Stepnoi, NN; Slesarev, IS; Kurchatov Institute of Atomic Energy (USSR)	Proceedings of the 1988 International Reactor Physics Conference. Supplement; 1988.; p. S.1-S.45.; International reactor physics conference.; Jackson Hole, WY (USA).; 18-22 Sep 1988.; American Nuclear Society.; LaGrange Park, IL (USA).; CONF-880911--Suppl.; American Nuclear Society, 555 N. Kensington Avenue, LaGrange Park, IL 60525.;	United States	1988
308	20008243	Interactions of core debris with concrete: The current state of the art, research, and remaining issues	Powers, DA; Sandia National Labs, Albuquerque, NM (USA)	[1988].; 20 p.; International symposium on severe accidents in nuclear power plants.; Sorrento (Italy).; 21-25 Mar 1988.; SAND--88-0861C.; CONF-880309--5.; Available from NTIS, PC A03/MF A01; 1 as DE88010832.; Portions of this document are illegible in microfiche products.;	United States	1988
309	21036035	Timing of the Three Mile Island Unit 2 core degradation as determined by forensic engineering	Henrie, JO; Hydrogen Control, Inc, Panguitch, UT (USA); EG and G Idaho, Inc, Idaho Falls, ID (USA)	[1988].; 8 p.; TMI-2 accident materials behavior, plant technology and recovery.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; EGG-M--88439.; CONF-881024--22.; Contract AC07-76ID01570.; NTIS, PC A02/MF A01 as DE90007045; OSTI; INIS; US Govt. Printing Office Dep.;	United States	1988

No.	登録番号	標題	著者情報	情報源	出版地	発行年
310	21020234	Modeling of thermal hydraulic behavior and fission product releases in degraded cores	Sharon, A; Ellison, PG; Henry, RE; Kenton, MA; Hammersley, RJ; Fauske and Associates, Inc, Burr Ridge, IL (USA); EG and G Idaho, Inc, Idaho Falls, ID (USA)	[1988].; 8 p.; International symposium on severe accidents in nuclear power plants IAEA - OECD/NEA.; Sorrento (Italy).; 21-25 Mar 1988.; EGG-M--32787.; CONF-880309--8; IAEA-SM--296/80.; Contract AC07-76ID01570.; Available from NTIS, PC A02/MF A01 as DE90001999; OSTI; INIS; US Govt. Printing Office Dep.;	United States	1988
311	21019331	Comparison of thermal and mechanical responses of the TMI-2 reactor vessel	Thinnes, GL; EG and G Idaho, Inc, Idaho Falls, ID (USA)	[1988].; 31 p.; Joint meeting of the European Nuclear Society and the American Nuclear Society.; Washington, DC (USA).; 30 Oct - 4 Nov 1988.; EGG-M--88438.; CONF-881011--42.; Contract AC07-76ID01570.; Available from NTIS, PC A03/MF A01 as DE90002032; OSTI; INIS; US Govt. Printing Office Dep.;	United States	1988
312	19002234	DCH-1: first direct containment heating experiment in the SURTSEY test facility	Tarbell, WW; Brockmann, JE; Pilch, M; Sandia National Labs, Albuquerque, NM	Proceedings of the US Nuclear Regulatory Commission fourteenth water reactor safety information meeting: Volume 6, TMI-2 analyses, Fission product release and transport in containment, Severe accident source term, Containment systems research/containment loads analysis; Feb 1987.; p. 233-258.; 14. water reactor safety information meeting.; Gaithersburg, MD (USA).; 27-31 Oct 1986.; NUREG/CP--0082-Vol.6.; NTIS, PC A18/MF A01 - US Govt. Printing Office. as TI87005860.;	United States	1987
313	19033043	Reactor risk reference document: Appendices J-O, Draft for comment	Nuclear Regulatory Commission, Washington, DC (USA) Office of Nuclear Regulatory Research	Feb 1987.; 513 p.; NUREG--1150-App.J-O-Vol.3.; Available from NTIS, PC A22/MF A01 - GPO as TI87900541.;	United States	1987
314	19036635	MELPROG-PWR/MOD0: A mechanistic code for analysis of reactor core melt progression and vessel attack under severe accident conditions	Camp, WJ; Young, MF; Tomkins, JL; Kelly, JE; Maudlin, PJ; Henninger, RJ; Sandia National Labs, Albuquerque, NM (USA); Nuclear Regulatory Commission, Washington, DC (USA) Div of Reactor Systems Safety; Science Applications International Corp, Albuquerque, NM (USA)	Apr 1987.; 155 p.; NUREG/CR--4909.; SAND--85-0237.; NTIS, PC A08/MF A01 - US Govt. Printing Office. as DE88000478.;	United States	1987
315	19051720	Ex-vessel defueling for TMI-2	Wolfgang, RJ; DiGiacomo, L; Ammenheuser, RN	Trans. Am. Nucl. Soc.; (1987).; Annual meeting of the American Nuclear Society.; Dallas, TX (USA).; 7-11 Jun 1987.; v. 54 p. 87-88.; CONF-870601--.;	United States	1987

No.	登録番号	標題	著者情報	情報源	出版地	発行年
316	19051739	A multifluid, multiphase flow and heat transfer model for the prediction of sweepout from a reactor cavity	Sienicki, JJ; Spencer, BW; Reactor Analysis and Safety Div, Argonne National Lab, 9700 South Cass Ave, Bldg 206, Argonne, IL 60439	Proceedings of the 4th Miami international symposium on multi-phase transport particulate phenomena (condensed papers); 1987.; p. 1.; 4. Miami international symposium on multi-phase transport and particulate phenomena.; Miami Beach, FL (USA).; 15-17 Dec 1986.; University of Miami.; Coral Gables, FL (USA).;	United States	1987
317	19078648	Results of the four PBF [Power Burst Facility] severe fuel damage tests	Osetek, DJ; EG and G Idaho, Inc, Idaho Falls (USA)	1987.; 26 p.; 15. water reactor safety information meeting.; Gaithersburg, MD (USA).; 26-30 Oct 1987.; EGG-M--21987.; CONF-8710111--35.; Available from NTIS, PC A03/MF A01; 1 as DE88006792.; Portions of this document are illegible in microfiche products.;	United States	1987
318	19078651	Analysis of the TMI-2 source range monitor during the TMI [Three Mile Island] accident	Wu, Horng-Yu; Baratta, AJ; Hsiao, Ming-Yuan; Bandini, BR; Pennsylvania State Univ, University Park (USA) Dept of Nuclear Engineering	Jun 1987.; 125 p.; EGG-TMI--7955.; Available from NTIS, PC A06; 3 as DE88006793.; Paper copy only, copy does not permit microfiche production.;	United States	1987
319	19002229	TMI-2 accident scenario update	Tolman, EL; Kuan, P; Broughton, JM; Idaho National Engineering Lab, Idaho Falls	Proceedings of the US Nuclear Regulatory Commission fourteenth water reactor safety information meeting: Volume 6, TMI-2 analyses, Fission product release and transport in containment, severe accident source term, Containment systems research/containment loads analysis; Feb 1987.; p. 39-54.; 14. water reactor safety information meeting.; Gaithersburg, MD (USA).; 27-31 Oct 1986.; NUREG/CP--0082-Vol.6.; NTIS, PC A18/MF A01 - US Govt. Printing Office. as TI87005860.;	United States	1987
320	19023801	The Chernobyl accident: Causes and consequences	Malinauskas, AP; Oak Ridge National Lab, TN (USA)	1987.; 9 p.; ANS topical conference on population exposure from nuclear fuel cycle.; Oak Ridge, TN (USA).; 15-18 Sep 1987.; CONF-8709104--3.; Available from NTIS, PC A02/MF A01; 1 as DE87014786.; Portions of this document are illegible in microfiche products.;	United States	1987
321	19046339	MELPROG analysis of the debris formation experiment DF-1	Heames, TJ; Gauntt, RO; Sandia National Labs, Albuquerque, NM (USA)	1987.; 32 p.; 15. water reactor safety information meeting.; Gaithersburg, MD (USA).; 26-30 Oct 1987.; SAND--87-2330C.; CONF-8710111--5.; Available from NTIS, PC A03/MF A01; 1 as DE88002411.; Portions of this document are illegible in microfiche products.;	United States	1987

No.	登録番号	標題	著者情報	情報源	出版地	発行年
322	19051719	TMI-2 RCS activity and solids loading from aggressive defueling techniques	Baston, VF; Hofstetter, KJ	Trans. Am. Nucl. Soc.; (1987).; Annual meeting of the American Nuclear Society.; Dallas, TX (USA).; 7-11 Jun 1987.; v. 54 p. 85-86.; CONF-870601--;	United States	1987
323	19084580	TMI-2 final plant assay: instrument options	Greenborg, J; Distenfeld, CH	Trans. Am. Nucl. Soc.; (1986).; American Nuclear Society annual meeting.; Reno, NV (USA).; 15-20 Jun 1986.; v. 52 p. 82-85.; CONF-860610--Summs.;	United States	1986
324	17062845	CEC sensitivity study of aerosol codes in LWR containment	Della Loggia, VE; Commission of the European Communities, Brussels (Belgium)	Source term evaluation for accident conditions; 1986.; p. 515-524.; International symposium on source term evaluation for accident conditions.; Columbus, OH (USA).; 28 Oct - 1 Nov 1985.; IAEA.; Vienna (Austria).; IAEA-SM--281/39.; 2 refs, 3 figs, 5 tabs.;	International Atomic Energy Agency (IAEA)	1986
325	18047345	Application of a direct-heating model to the Sandia SURTSEY tests	Pong, LC; Huhtiniemi, I; Corradini, ML; Univ of Wisconsin, Madison	Trans. Am. Nucl. Soc.; (1986).; American Nuclear Society and Atomic Industrial Forum joint meeting.; Washington, DC (USA).; 16-21 Nov 1986.; v. 53 p. 558-559.; CONF-861102--;	United States	1986
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349	19023574	Review of the main results of the SASCHA program on fission product release under core melting conditions	Albrecht, H; Wild, H; Kernforschungszentrum, Karlsruhe, West Germany	American Nuclear Society meeting on fission-product behavior and source term research: proceedings; Jul 1985.; p. 3.1-3.14.; Topical meeting on fission product behavior and source term research.; Snowbird, UT (USA).; 15-19 Jul 1984.; EPRI-NP--4113-SR.; Research Reports Center, Box 50490, Palo Alto, CA 940303.;	United States	1985
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365	16027482	Results of the in-pile degraded core coolability experiments: DCC-1 and DDC-2	Boldt, KR; Kuenstler, PA Jr; Schmidt, TR; Sandia National Labs, Albuquerque, NM (USA)	1984.; 11 p.; International meeting on thermal nuclear reactor safety.; Karlsruhe (Germany, F.R.); 10-14 Sep 1984.; SAND--84-0522C.; CONF-840914--12.; Available from NTIS, PC A02/MF A01; 1 as DE84017004.;	United States	1984
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371	17069369	The MEDICI reactor cavity model	Bergeron, KD; Trebickock, W; Sandia National Laboratories, Albuquerque, NM	Light water reactor severe accident evaluation; Aug 1983.; p. 5.6-1-5.6-7.; International meeting on light-water reactor severe accident evaluation.; Cambridge, MA (USA).; 28 Aug - 1 Sep 1983.; Stone and Webster Engineering Corporation.; Boston, MA (USA).;	United States	1983
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374	17069340	Core debris quenching heat transfer rates under top- and bottom-reflood conditions	Ginsberg, T; Klages, J; Klein, J; Sanborn, Y; Schwarz, CE; Tutu, NK; Department of Nuclear Energy, Brookhaven National Laboratory, Upton, NY	Light water reactor severe accident evaluation; Aug 1983.; p. 18.7-1-18.7-6.; International meeting on light-water reactor severe accident evaluation.; Cambridge, MA (USA).; 28 Aug - 1 Sep 1983.; Stone and Webster Engineering Corporation.; Boston, MA (USA).;	United States	1983
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377	14792571	Effect of debris bed pressure, particle size, and distribution on degraded nuclear reactor core coolability	Squarer, D; Pieczynski, AT; Hochreiter, LE; Westinghouse Electr Corp, Pittsburgh, Pa, USA	Nucl. Sci. Eng.; (Jan 1982).; v. 80(1) p. 2-13.;	United States	1982
378	14737489	Establishment of a permanently coolable state	Fauske, HK; Grolmes, MA; Henry, RE; Fauske and Assoc, Willowbrook, IL 60521	Trans. Am. Nucl. Soc.; (1981).; ANS winter meeting.; San Francisco, CA (USA).; 29 Nov - 4 Dec 1981.; v. 39 p. 368-369.; CONF-811103--.; Published in summary form only.;	United States	1981